

Targets for Cyclotron Production of Tc-99m

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Introduction: The measured yields of direct ^{99m}Tc production via $^{100}\text{Mo}(\text{p},2\text{n})^{99m}\text{Tc}$ suggest that ^{99m}Tc can be produced in quantities sufficient for supplying regional radiopharmacies^{i, ii, iii}, thereby reducing our reliance on reactor-derived ^{99}Mo . Cyclotron- and generator-produced ^{99m}Tc -radiopharmaceuticals were shown to be radionuclidically, chemically and biologically equivalent, giving matching images and identical kinetic and biodistribution patterns in animals, indicating that a medical cyclotron can produce USP-compliant ^{99m}Tc -radiopharmaceuticals for nuclear imaging procedures.^{iv, v} In this work, several different ^{100}Mo target configurations were investigated and thick target yields were measured, validating the production of clinically useful quantities of ^{99m}Tc on a medical cyclotron.

Target Holders: Two different solid target holders were used to measure the thick target yields of the $^{100}\text{Mo}(\text{p},2\text{n})^{99m}\text{Tc}$ nuclear reaction. The straight 90° target holder has a heat removal capacity of 1.5 kW and while the 30° tilted solid target holder has a heat removal capacity of 3.0 kW. Two different solid target holders (Advanced Cyclotron Systems Inc., Richmond, BC, Canada) were installed on three compact medical cyclotrons (TR-19, Cross Cancer Institute, Edmonton, AB, TR-19 Centre Hospitalier Universitaire de Sherbrooke, Sherbrooke QC, Canada, GE PETrace, Lawson Health Research Institute, London ON, Canada).



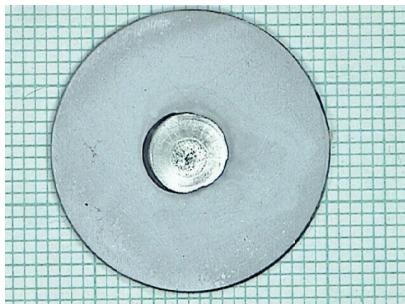
30° Solid Target Holder



Straight Solid Target Holder

^{100}Mo Targetry. Molybdenum has been a metal of choice in accelerator targetry for several decades. With a high melting point, good thermal conductivity and chemical stability, molybdenum is nearly an ideal material for manufacturing high power targets. While a number of low and medium current cyclotron targets that use natural and enriched molybdenum isotopes have been developed and used for production of technetium isotopes: ^{94}Tc , ^{96}Tc and ^{99m}Tc ^{vi}, a reliable process for preparation of enriched molybdenum targets has not yet been implemented. A number of standard target manufacturing techniques are being evaluated: melting, sintering, pressing/pelletizing, rolling, plating from solutions or molten salts, formation of low melting temperature Mo alloys, brazing or soldering ^{100}Mo to a target substrate, coating molybdenum with a protective layer, development of a thick target, plasma sputtering and other coating techniques.

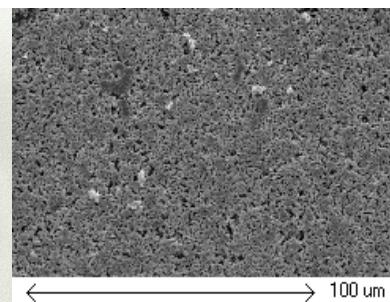
Mo Target Preparation: Between 100-450 mg natural and enriched ^{100}Mo (99.5%) were pressed into 6 and 9.5 mm pellets at between 25,000 N and 100,000 N. The pellets were sintered in wet or dry hydrogen at 800-900°C, and subsequently mounted into a tantalum substrate, either by pressing or arc melting or electron beam melting at currents between 40-70 mA with different sweeping / focusing patterns.



1. Arc Melted Mo in tantalum



2. Pressed Mo in Ta (EOB)



3. SEM of pressed Mo

^{99m}Tc Production: ^{99m}Tc was produced via the $^{100}\text{Mo}(p,2n)^{99m}\text{Tc}$ nuclear reaction on a 19 MeV medical cyclotron using an incident proton energy of 15-17 MeV at current between 14-52 μA . After bombardment targets were subjected to electrochemical dissolution, ^{99m}Tc was purified by ion-exchange chromatography and recovered as pertechnetate.



Electron beam melting of Mo to Tatarget substrate

Results: Up to 44.7 GBq (1.2 Ci) (EOB) of ^{99m}Tc was produced after a 6-h bombardment at 16.4 MeV and 39 μA . This corresponds to a thick target production yield of 0.25 GBq/ $\mu\text{A}/\text{h}$ (6.8 mCi/ $\mu\text{A}/\text{h}$) and 2.3 GBq/ μA (63 mCi/ μA) at saturation and is in good agreement with literature data.^{i, ii, iii} The radionuclide purity of the cyclotron-produced ^{99m}Tc was >99.99%, as assessed by γ spectroscopy, exceeding USP requirements for generator-based ^{99m}Tc . The content of other technetium isotopes was measured after allowing sufficient time (4 days) for ^{99m}Tc decay and was below USP requirements of 0.01% for generator-produced ^{99m}Tc . No other radionuclidic impurities were found. The radiochemical purity of cyclotron-produced $^{99m}\text{TcO}_4^-$ was >99.5%, well above the USP requirement of 95%.

Conclusion: This study confirms that clinically useful quantities of ^{99m}Tc can be produced on medical cyclotrons installed worldwide. Extrapolating these results to the optimal energy of 22-24 MeV indicates that over 2 TBq of ^{99m}Tc can be produced daily for regional distribution on a high-current medium-energy cyclotron. Implementing networks of high-current medium energy cyclotrons would reduce reliance on nuclear reactors and attenuate the negative consequences associated with the use of fission technology.

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